

Review of the Grischuk and Sachin Gravitational Wave Generator Via Tokamak Physics

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Abstract

Using the Grischuk and Sachin amplitude for the GW generation due to plasma in a toroid, we generalize this result for Tokamak physics. We obtain evidence for strain values of up to $h_{2nd-term} \sim 10^{-23} - 10^{-24}$ in the center of a Tokamak, with a minimum value of $h \sim 10^{-26}$ five meters above the center of the Tokamak. These values are an order of magnitude sufficient to allow for possible detection of gravitational waves, in the coming decade, according to Chinese researchers who are trying to do such in Hefei, PRC. The critical breakthrough is in utilizing a burning plasma drift current which relies upon a thermal contribution to an electric field. The gravitational wave amplitude would be detectable in part also due to $n_{ion} \cdot \tau_E > .5 \times 10^{20} \cdot m^{-3} \cdot sec$, where the n_{ion} is the numerical ion density, usually about $10^{20} \cdot m^{-3}$, i.e. about one out of a million of the present atmospheric pressure, whereas τ_E is a confinement time value for the Tokamak plasma, here at least .5 seconds. This value as given above is one given by Wesson in his Tokamak classic reference. This is the threshold for plasma fusion burning and the temperature so obtained is the main driver for how one could conceivably detect GW of amplitude as low as

$$\left[h_{2nd-term} \Big|_{T_{Temp} \geq 100 KeV} \right]_{5-meters-above-Tokamak} \sim 10^{-25} \text{ five meters above the center of the Tokamak.}$$

Key words: Tokamak physics, confinement time (of Plasma), space-time GW amplitude, Drift current

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1. Introduction

In 1975, in [1], Russian physicists Grishchuk and Sachin obtained the following amplitude of a Gravitational wave (GW) in a plasma which is given as

$$A(\text{amplitude-GW}) = h \sim \frac{G}{c^4} \cdot E^2 \cdot \lambda_{GW}^2 \quad (1)$$

Here, E is the electric field whereas λ_{GW} is the Gravitational wavelength for GW generated by the Tokamak, in our model. Note, if $\omega_{GW} \sim 10^6 Hz \Rightarrow \lambda_{GW} \sim 300 \text{ meters}$, so we will be assuming a baseline of the order of

$\omega_{GW} \sim 10^9 Hz \Rightarrow \lambda_{GW} \sim .3 \text{ meters}$, of about a foot, as a start for GW detection above the Tokamak. What we will be doing is to examine the would be electric field, and this in ways different from the initial Ohms law

$$J = \sigma \cdot E \quad (2)$$

In order to obtain a suitable electric field, which is to be detected via 3DSR technology [2] we will use a generalized Ohm's law as given by Wesson [3] as seen in page 146, where E and B are for electric and magnetic fields, and v is for velocity.

$$E = \sigma^{-1}J - v \times B \quad (3)$$

What we will be looking for is an application for radial free electric fields being applied, namely [3] Wesson, page 120

$$n_j e_j \cdot (E_r + v_{\perp j} B) = -\frac{dP_j}{dr} \quad (4)$$

Here, n_j = ion density, jth species, e_j = ion charge, jth species, E_r = radial electric field, $v_{\perp j}$ = perpendicular velocity, of jth species, B = magnetic field, and P_j = pressure, jth species. The results of Eq. (3) and Eq. (4) are

$$\frac{G}{c^4} \cdot E^2 \cdot \lambda_{GW}^2 \sim \frac{G}{c^4} \cdot \left[\frac{Const}{R} \right]^2 \cdot \lambda_{GW}^2 + \frac{G}{c^4} \cdot \left[\frac{J_b}{n \cdot e} + v_R \right]^2 \cdot \lambda_{GW}^2 = (1^{st}) + (2^{nd}) \quad (5)$$

Here, the 1st term is due to $\nabla \times E = 0$, and the 2nd term is due to $E_n = \frac{dP_j}{dx_n} \cdot \frac{1}{n_j \cdot e_j} - (v \times B)_n$ with the 1st term

generating $h \sim 10^{-38} - 10^{-30}$ in terms of GW amplitude strain 5 meters above the Tokamak, whereas the 2nd term has an $h \sim 10^{-26}$ in terms of GW amplitude above the Tokamak. The article has contributions from amplitude from the 1st and 2nd terms separately. The second part will be tabulated separately from the first contribution assuming a minimum temperature of $T = Temp \sim 10KeV$ as from Wesson [3]

2. GW h strain values when the first term of Eq.(5) is used for different Tokamaks

We now look at what we can expect with the simple Ohm's law calculation for strain values. This is work which the Author with Gary Stevenson and Amara Angelica did in late 2012. As it is, the effort lead to non usable GW amplitude values of up to $h \sim 10^{-38} - 10^{-30}$ for GW wave amplitudes 5 meters above a Tokamak, and $h \sim 10^{-36} - 10^{-28}$ in the center of a Tokamak. I.e. this would be using Ohm's law and these are sample values of the Tokamak generated GW amplitude, using the first term of Eq. (5) and obtaining the following value

$$h_{First-term} \sim \frac{G}{c^4} \cdot E^2 \cdot \lambda_{GW}^2 \sim \frac{G}{c^4} \cdot \left[\frac{J}{\sigma} \right]^2 \cdot \lambda_{GW}^2 \quad (5a)$$

We summarize the results of such in our first table as given for when $\omega_{GW} \sim 10^9 Hz \Rightarrow \lambda_{GW} \sim .3 meters$ and with conductivity $\sigma(tokamak - plasma) \sim 10 \cdot m^2/sec$ and with the following denouncement. What we observe are a range of Tokamak values which are, even in the case of ITER (not yet built) beyond the reach of any technological detection devices which are conceivable in the coming decade. This table and its results, assuming fixed conductivity values $\sigma(tokamak - plasma) \sim 10 \cdot m^2/sec$ as well as $\lambda_{GW} \sim .3 meters$ is why the author, after due consideration completed his derivation of results as to the 2nd term of Eq. (5) which lead to even for when considering the results for the Chinese Tokamak in Hefei to have

$$h_{Second-term} \sim \frac{G}{c^4} \cdot E^2 \cdot \lambda_{GW}^2 \sim \frac{G}{c^4} \cdot \left[\frac{J_b}{n \cdot e} + v_R \right]^2 \cdot \lambda_{GW}^2 \text{ values } 10,000 \text{ larger than the results in ITER due to Eq.(5a).}$$

Table 1: Values of strain at center of Tokamak, and 5 meters above Tokamak:

$\lambda_{GW} \sim .3$ meters , $\sigma(\text{tokamak} - \text{plasma}) \sim 10 \cdot m^2/\text{sec}$, using Eq.6 above for Amplitude of GW.

Experiment	Site/ location	Plasma current, in (mili-Amps) MA	Strain, h, in center of the Tokamak	Strain, h, 5 meters above the center of the Tokamak
JET	Culan (UK)	5-7	$h \sim 10^{-31}$	$h \sim 10^{-33}$
ASDEX	Garching (GER)	2	$h \sim 10^{-32}$	$h \sim 10^{-34}$
DIII-D	San Diego (USA)	1.5-3	$h \sim 10^{-32}$	$h \sim 10^{-34}$
HL-2A	Chengdu (PRC)	.45	$h \sim 10^{-34}$	$h \sim 10^{-36}$
HT-7U	Hefei (PRC)	.25	$h \sim 10^{-36}$	$h \sim 10^{-38}$
ITER(planned)	Saint Paul Les-Durance (FR)	15	$h \sim 10^{-28} - 10^{-29}$	$h \sim 10^{-30} - 10^{-31}$

What makes it mandatory going to the 2nd term of Eq. (5) is that even in the case of ITER, 5 meters above the Tokamak ring, that the GW amplitude is 1/10,000 the size of any reasonable GW detection device, and this including the new 3DSR technology [2]. Hence, we need to come up with a better estimate, which is what the 2nd term of Eq.(5) is about which is derived in the next section

3 . Enhancing GW strain Amplitude via utilizing a burning Plasma drift current: Eq.(4)

The way forward is to go to Wesson, [3], page 120 and to look at the normal to the surface induced electric field contribution

$$E_n = \frac{dP_j}{dx_n} \cdot \frac{1}{n_j \cdot e_j} - (v \times B)_n \quad (6)$$

If one has for v_R as the radial velocity of ions in the Tokamak from the center of a Tokamak, to its radial distance, R, from the center, and B_θ being the direction of a magnetic field in the ‘face’ of a Toroid containing the Plasma, in the angular θ direction from a minimal toroid radius of $R = a$, with $\theta = 0$, to $R = a + r$ with $\theta = \pi$, one has v_R for radial drift velocity of ions in the Tokamak, and B_θ having a net approximate value of:

$$(v \times B)_n \sim v_R \cdot B_\theta \quad (7)$$

Also, as a first order approximation: From reference [3] page 167 the spatial change in pressure denoted

$$\frac{dP_j}{dx_n} = -B_\theta \cdot j_b \quad (8)$$

Here from Wesson[3], on page 167 the drift current, using $\xi = a/R$, and drift current j_b for Plasma charges, i.e.

$$j_b \sim -\frac{\xi^{1/2}}{B_\theta} \cdot T_{Temp} \cdot \frac{dn_{drift}}{dr} \quad (9)$$

Then one has

$$B_\theta^2 \cdot (j_b/n_j \cdot e_j)^2 \sim \frac{B_\theta^2}{e_j^2} \cdot \frac{\xi^{1/4}}{B_\theta^2} \cdot \left[\frac{1}{n_{drift}} \cdot \frac{dn_{drift}}{dr} \right]^2 \sim \frac{\xi^{1/4}}{e_j^2} \cdot \left[\frac{1}{n_{drift}} \cdot \frac{dn_{drift}}{dr} \right]^2 \quad (10)$$

Now, the behavior of the numerical density of ions, can be given as follows, namely growing in the radial direction, then

$$n_{drift} = n_{drift}|_{initial} \cdot \exp[\tilde{\alpha} \cdot r] \quad (11)$$

This exponential behavior then will lead to the 2nd term in Eq.(5) having in the center of the Tokamak, for an ignition temperature of $T_{Temp} \geq 10KeV$ a value of

$$h_{2nd-term} \sim \frac{G}{c^4} \cdot B_\theta^2 \cdot (j_b/n_j \cdot e_j)^2 \cdot \lambda_{GW}^2 \sim \frac{G}{c^4} \cdot \frac{\xi^{1/4} \tilde{\alpha}^2 T_{Temp}^2}{e_j^2} \cdot \lambda_{GW}^2 \sim 10^{-25} \quad (12)$$

Note that by page 11 of reference [3], Wesson, that there is a critical ignition temperature at its lowest point of the curve in the figure on page 11 having $T_{Temp} \geq 30KeV$ as an optimum value of the Tokamak ignition temperature for $n_{ion} \sim 10^{20} m^{-3}$, with a still permissible temperature value of $T_{Temp}|_{safe-upper-bound} \approx 100KeV$ with a value of $n_{ion} \sim 10^{20} m^{-3}$, due to from page 11, [3] the relationship of Eq.(13), where τ_E is a Tokamak confinement of plasma time of about 1-3 seconds, at least due to

$$n_{ion} \cdot \tau_E > .5 \times 10^{20} \cdot m^{-3} \cdot sec \quad (13)$$

Also, if one is using the curve on page 11 of [3], that $T_{Temp}|_{safe-upper-bound} \approx 100KeV$, then one could have at the center of a Tokamak, i.e. even the Hefei based PRC Tokamak

$$h_{2nd-term}|_{T_{Temp} \geq 100KeV} \sim \frac{G}{c^4} \cdot \frac{\xi^{1/4} \tilde{\alpha}^2 T_{Temp}^2}{e_j^2} \cdot \lambda_{GW}^2 \sim 10^{-23} \quad (14)$$

This would lead to, for a GW reading 5 meters above the Tokamak, then lead to for then the Hefei PRC Tokamak

$$\left[h_{2nd-term}|_{T_{Temp} \geq 100KeV} \right]_{5-meters-above-Tokamak} \sim \frac{G}{c^4} \cdot \frac{\xi^{1/4} \tilde{\alpha}^2 T_{Temp}^2}{e_j^2} \cdot \lambda_{GW}^2 \sim 10^{-25} \quad (15)$$

4. Details of the model in terms of terms of adding impurities to the Plasma to get a longer confinement time (possibly to improve the chances of GW detection).

We add this detail in, due to a question raised by Dr. Li who wished for longer confinement times for the Plasma in order to allegedly improve the chances of GW detection for a detector 5 meters above the Tokamak in Hefei. What Wesson stated is that the confinement time may be made proportional to the numerical density of argon/ neon seeded to the plasma. See page 180 of [3]. This depends upon the nature of the Tokamak, but it is a known technique, and is suitable for analysis, depending upon the specifics of the Tokamak. I.e. this is a detail Dr. Li can raise with his co workers in Hefei, PRC in 2014.

5. Conclusion. GW generation due to the Thermal output of Fusion in a Tokamak, and not due to E and B field currents.

Further elaboration of this matter lies in the viability of the expression derived , namely Eq. (15) repeated

$$\left[h_{2nd-term} \Big|_{T_{Temp} \geq 100 KeV} \right]_{5-meters-above-Tokamak} \sim \frac{G}{c^4} \cdot \frac{\xi^{1/4} \tilde{\alpha}^2 T_{Temp}^2}{e_j^2} \cdot \lambda_{GW}^2 \quad (15)$$

The importance of the formulation is in the explicit importance of temperature. i.e. a temperature range of at least $10KeV \leq T_{Temp} \leq 100KeV$. In making this range for Eq.(15) care must also be taken to obtain a sufficiently long confinement time for the fusion plasma in the Tokamak of at least 1 second or longer, and this is a matter of applied engineering dependent upon the instrumentation of the Tokamak in Hefei, PRC. The author hopes that in 2014, there will be the beginning of confirmation of this process so that some studies may commence. If so, then the next question will be finding if the instrumentation of [2] can be utilized and developed. This is expected to be extremely difficult, but the Tokamak fusion process may allow for falsifiable testing and eventual verification.

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